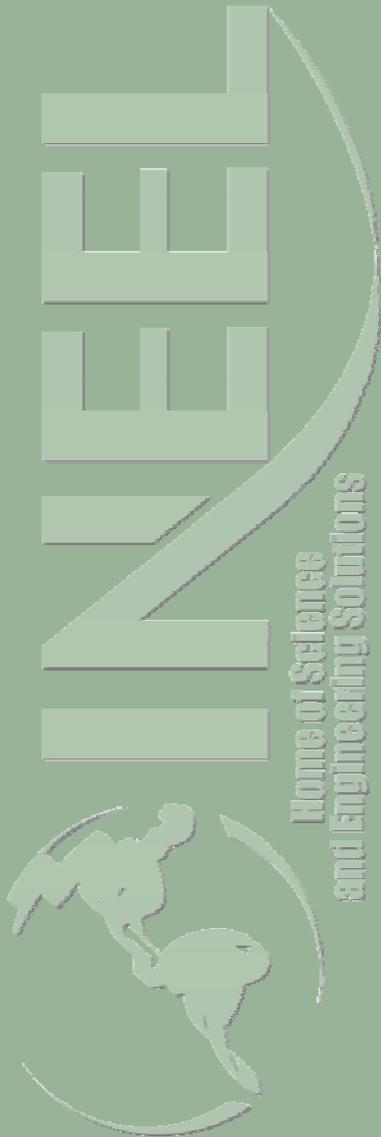


*Idaho National Engineering and Environmental Laboratory*

# ***Modeling of Supercritical Pressurized Water Reactors with SCDAP/RELAP5-3D***

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# Presentation Overview

- *Basic design of Supercritical Pressure Water Reactor (SCWR).*
- *Comparison of properties of Zircaloy fuel cladding and a candidate SCWR fuel cladding (Alloy MA956).*
- *Comparison of structural behavior of Zircaloy clad and Alloy MA956 clad fuel rods during heatup representative of post-blowdown period of LOCA.*
- *Comparison of cladding temperature behavior during flow reduction accident in SCWR using different models for convective heat transfer.*
- *Conclusions.*

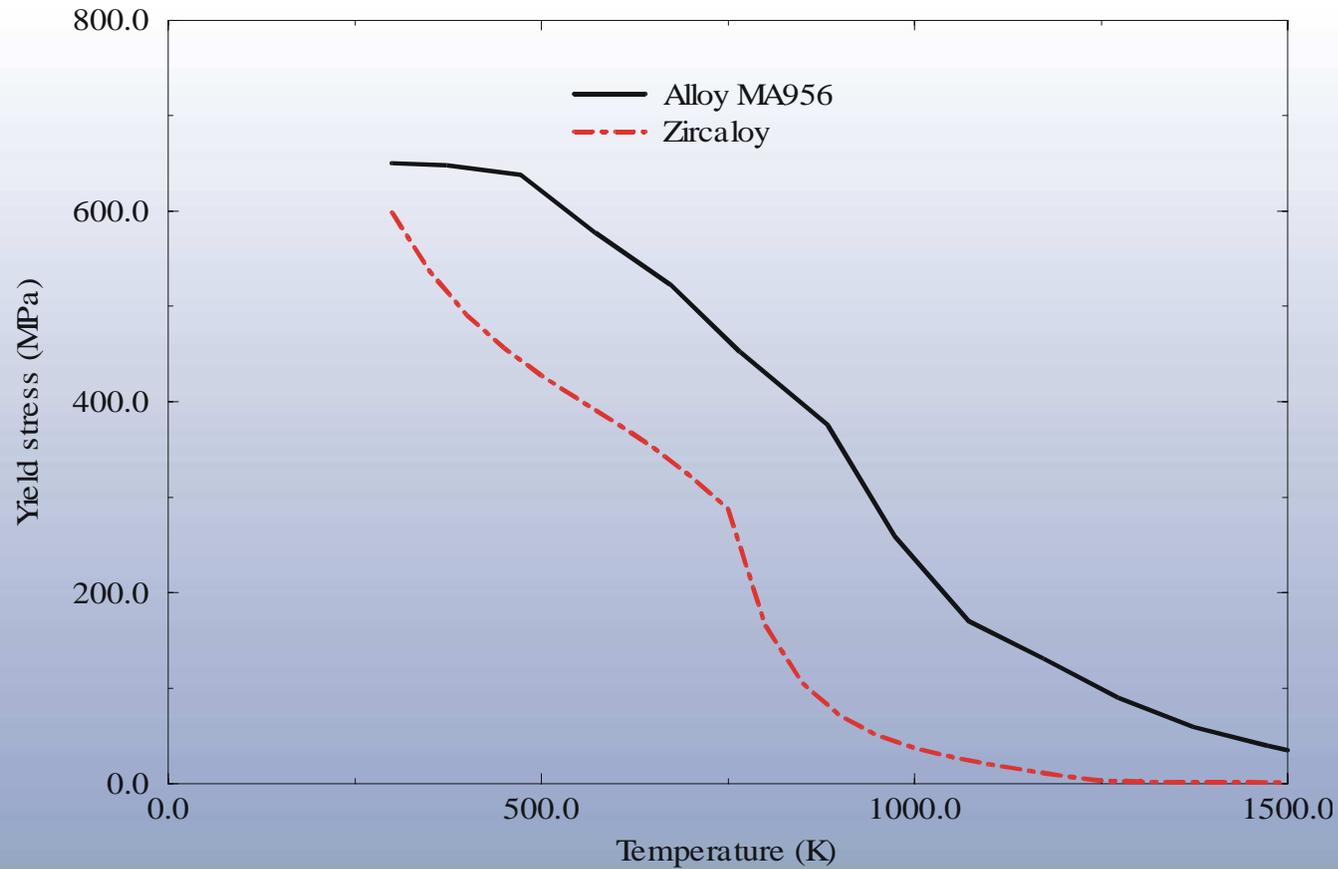
# ***SCWRs differ from Generation III LWRs in several ways to achieve 25% greater thermal efficiency***

- *Coolant pressure of 25.0 MPa instead of 15 MPa in PWR and 7 MPa in BWR.*
- *Coolant core exit temperature of 800 K instead of 560 K in SBWR.*
- *Cladding composed of high temperature alloys such Fe-based MA956 and Ni-based Inconel 718 instead of Zircaloy.*
- *Once-through direct flow of all coolant (working fluid) from main feedwater pumps to reactor core and no steam separators, dryers, or recirculation lines.*
- *Higher fill gas pressure.*

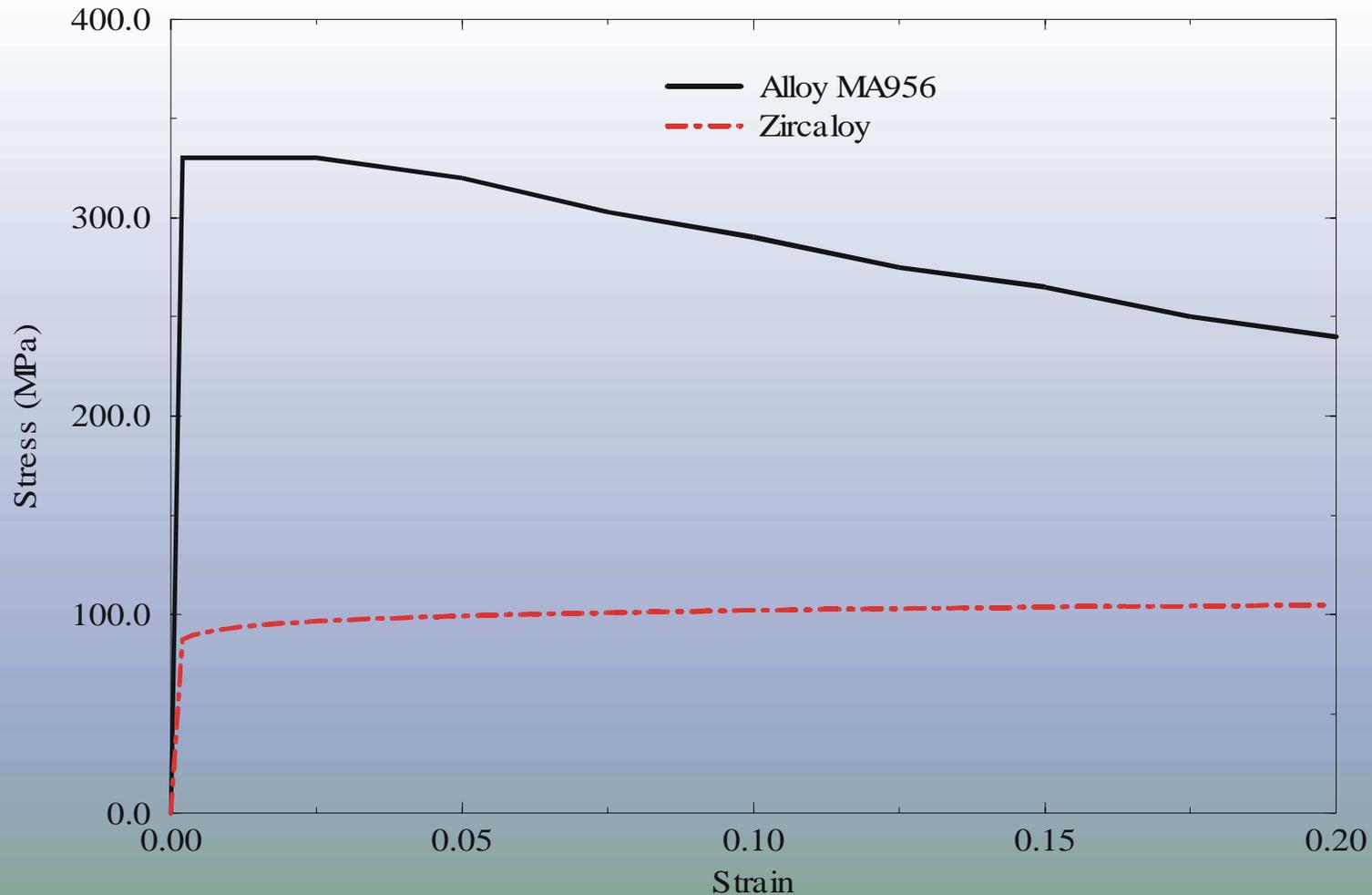
# ***Proposed cladding materials for SCWRs have high temperature capability and oxidation resistance***

- *Alloy MA956 has excellent strength at high temperatures due to dispersion of yttrium oxide ( $Y_2O_3$ ).*
- *Composition of MA956: 74.5wt% Fe, 20wt% Cr, 4.5wt%Al, 0.5wt%  $Y_2O_3$ .*
- *Composition of Inconel 718: 52.6wt% Ni, 18wt%Cr, 24wt% Fe, 3wt% Mo.*
- *Other Ferritic-Martensitic steels; T91 (9Cr-1Mo-V), A-21 (9Cr-TiC), new stainless steels; HT-UPS*

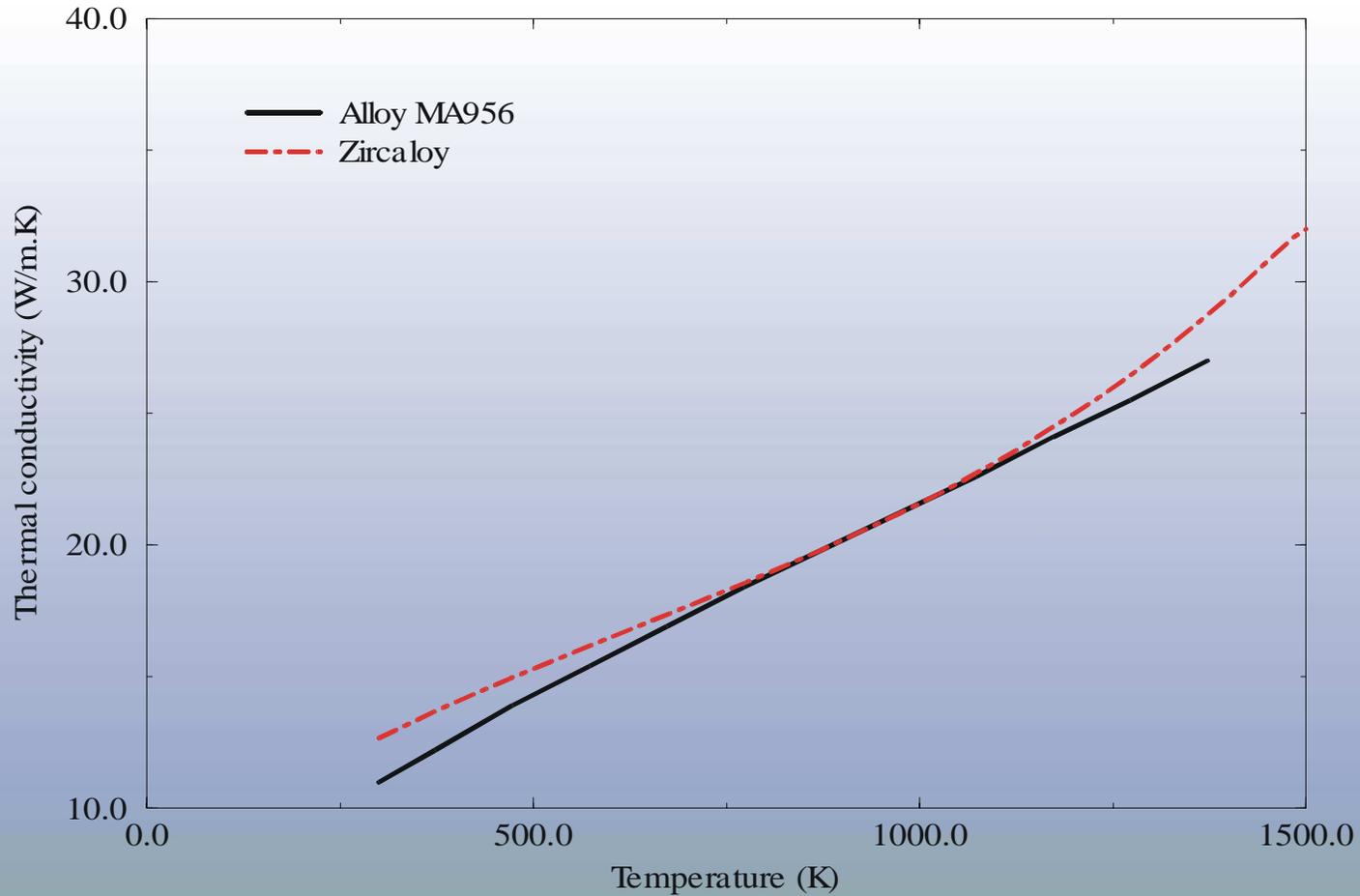
## **MA956 has small decrease in strength as temperature increases from 300 K to 500 K**



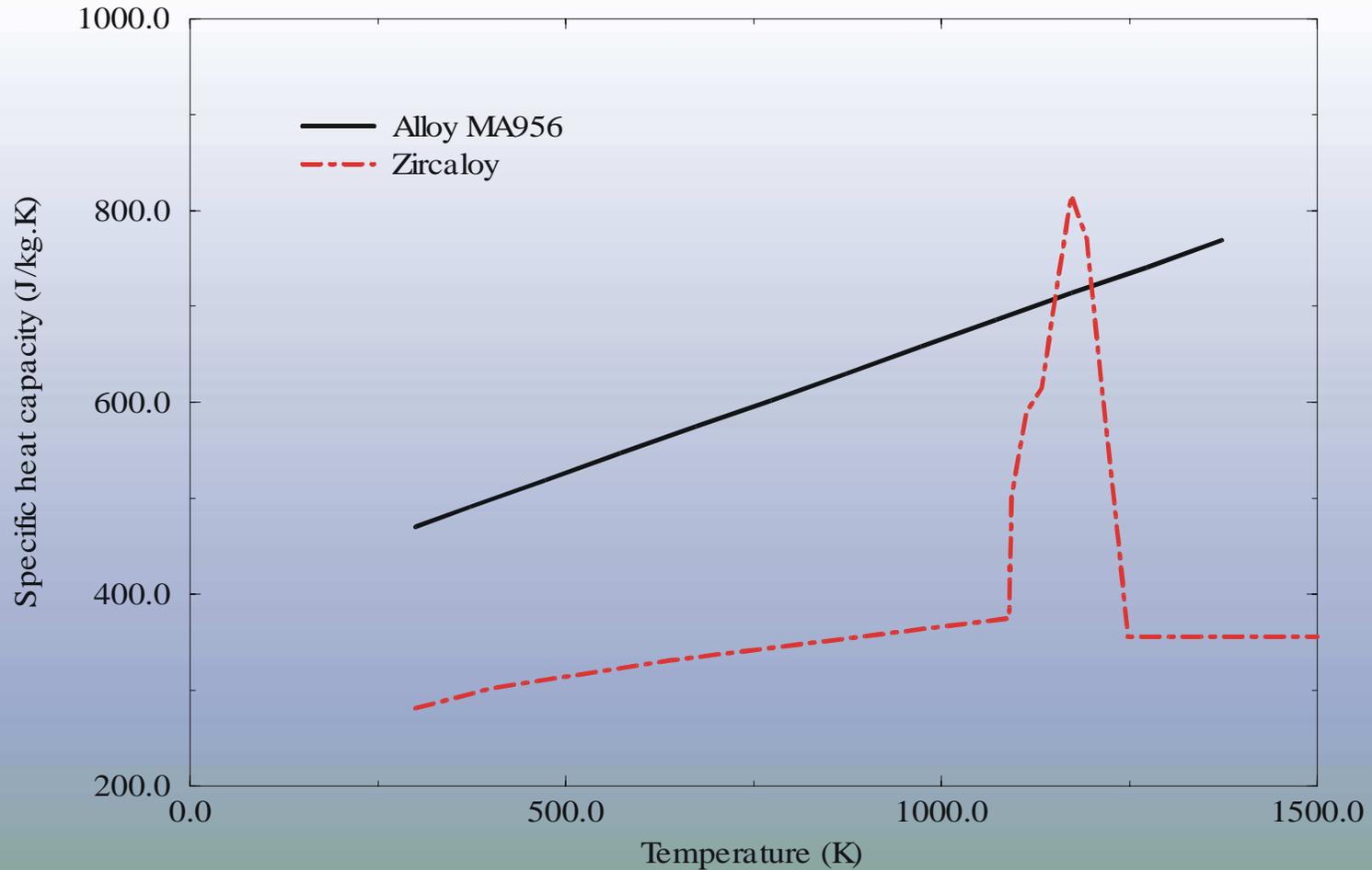
**MA956 at 873 K has about three times the strength of Zircaloy and similar ductility**



## ***Thermal conductivities of MA956 and Zircaloy are similar***



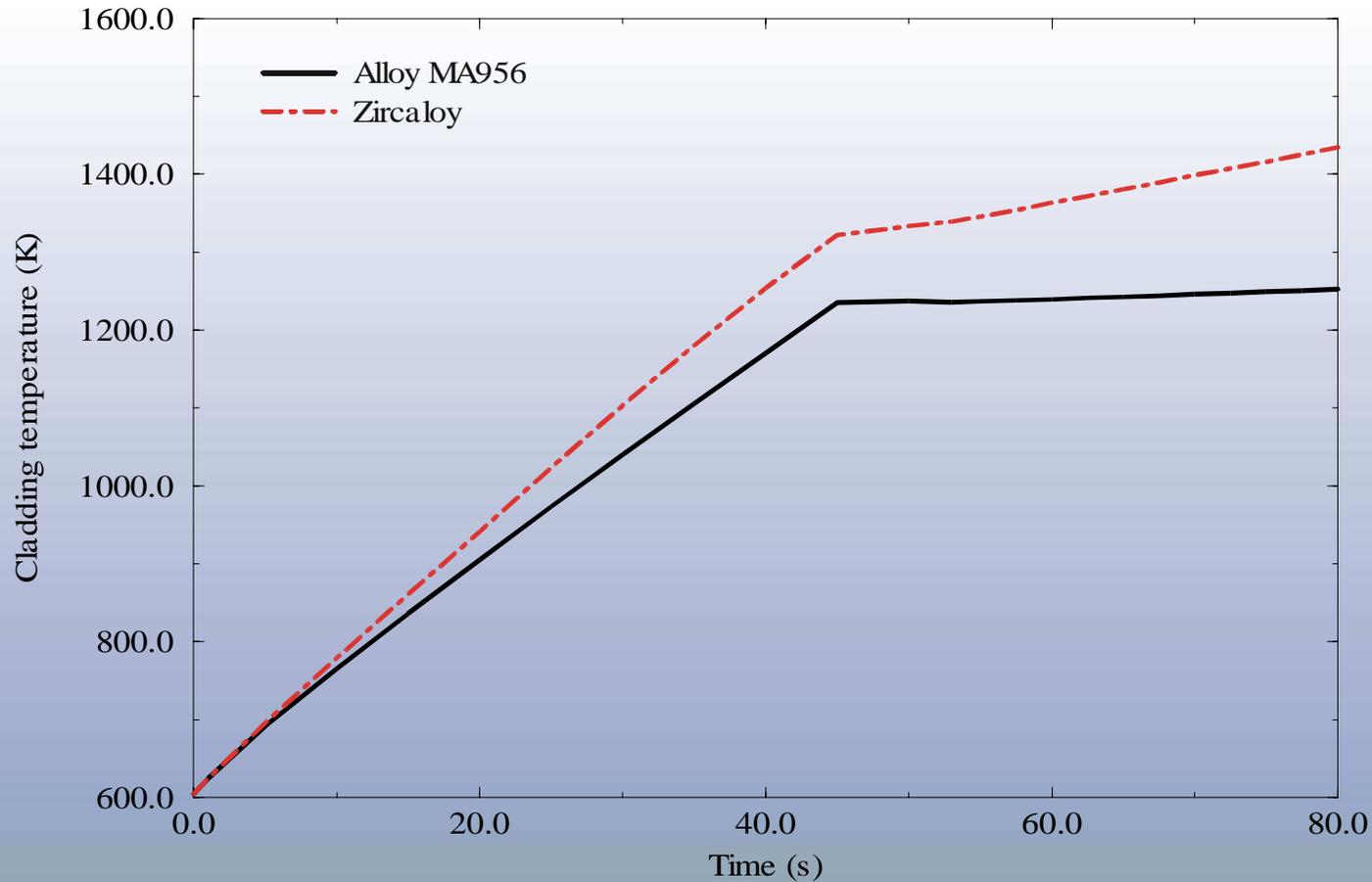
## Stored energy of MA956 cladding is greater than that of Zircaloy cladding



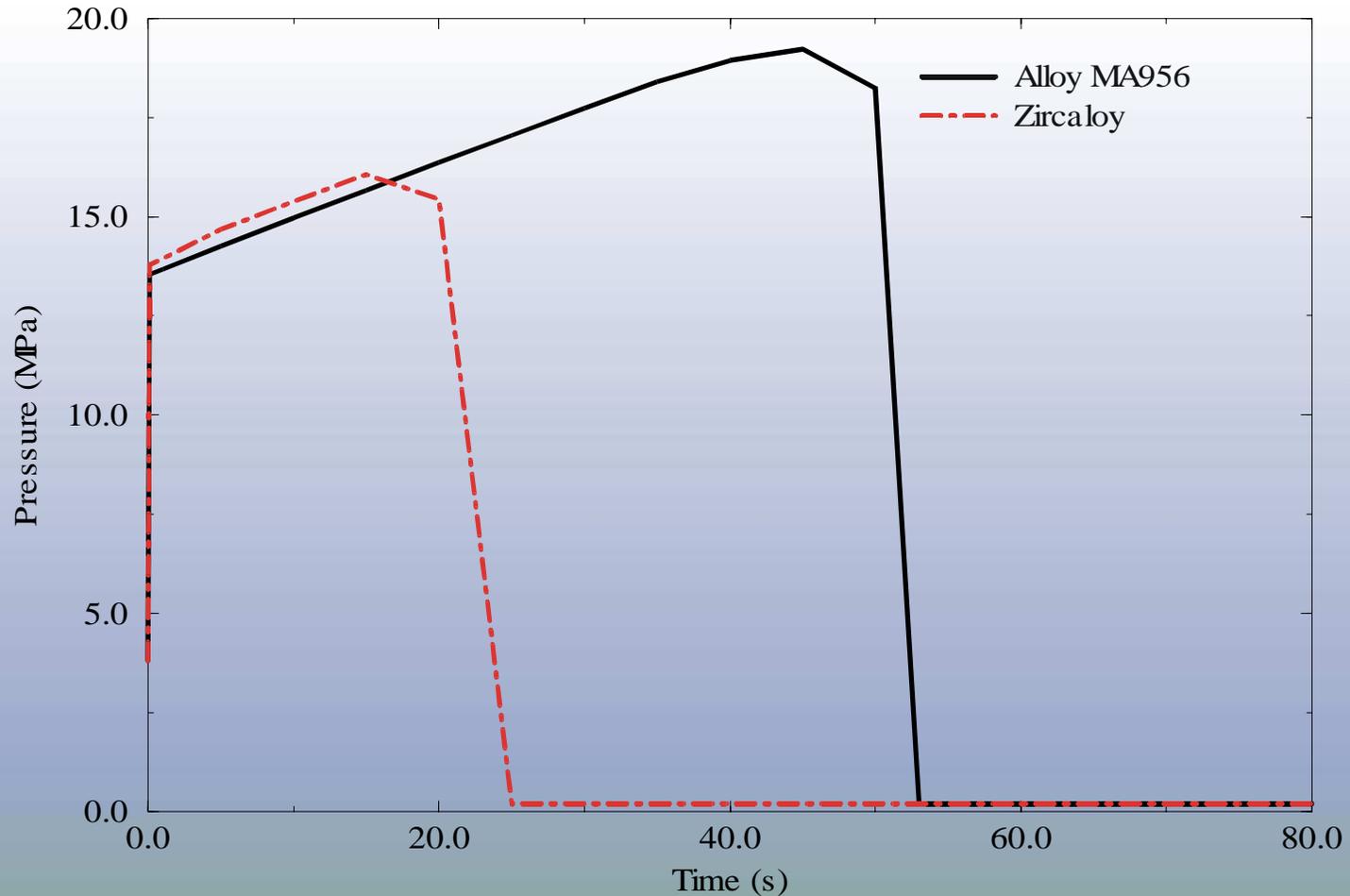
***Coolant conditions similar to post-blowdown phase of LOCA used to compare deformation behavior of MA956 and Zircaloy***

- *Fuel rod power corresponding with decay heat a few seconds after reactor scram.*
- *Coolant pressure of 0.2 MPa.*
- *Steam flow rate of 0.054 kg/m<sup>2</sup>.s.*
- *Temperature of steam at inlet to fuel bundle of 500 K.*

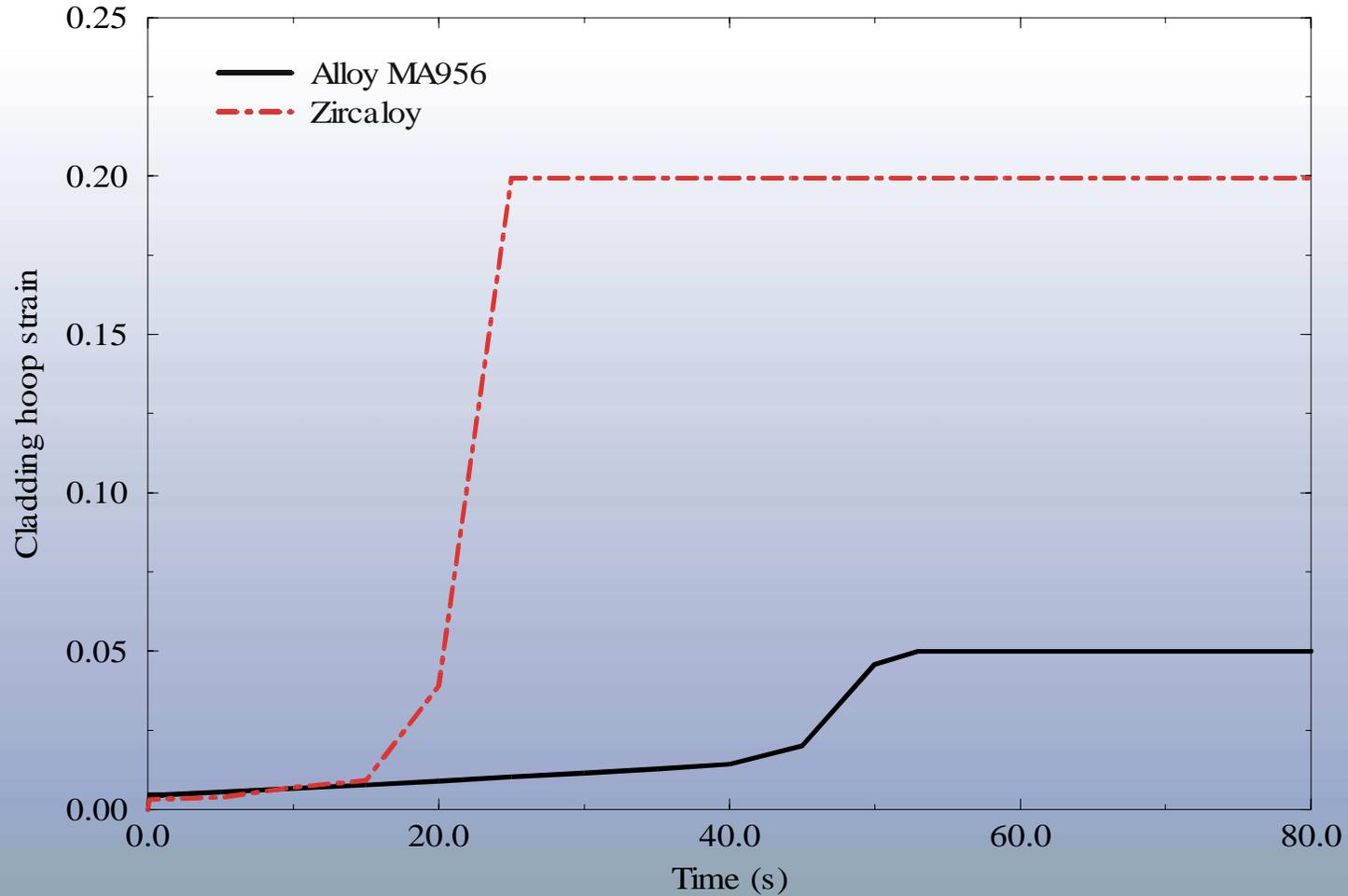
## Temperature of Zircaloy cladding during heatup period of LOCA greater than that of MA956 due to oxidation



**Initial pressure in SCWR fuel rods expected to be near EOL internal pressure in LWR fuel rods**



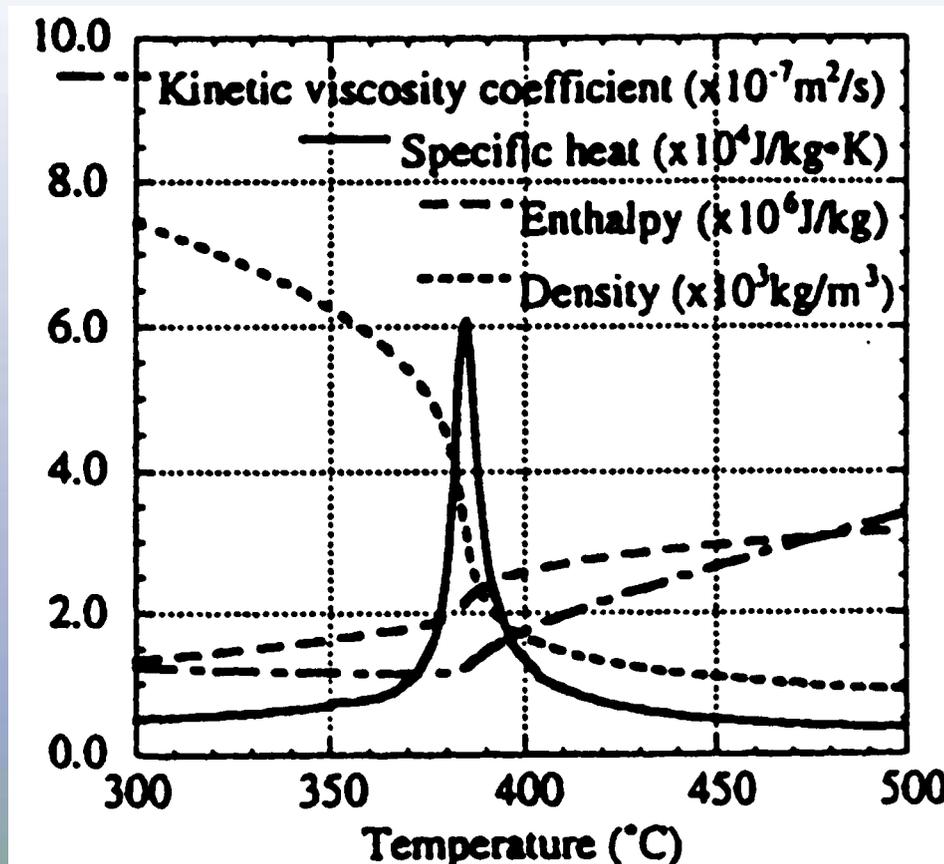
## SCWR fuel rods during LOCA may not balloon as much as LWR fuel rods



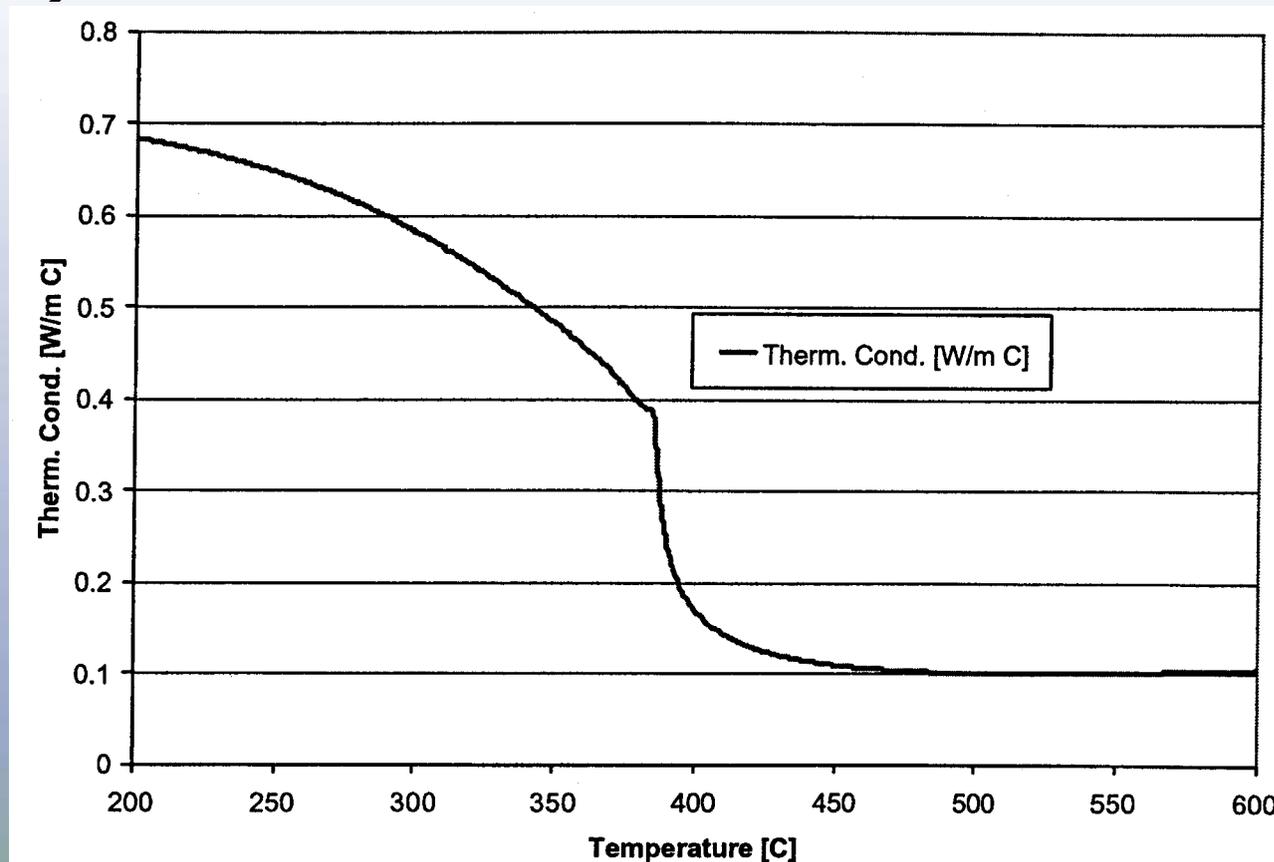
# ***Convective heat transfer in SCWR investigated using Power-Coolant Mismatch transient***

- *One bundle of fuel rods modeled.*
- *Power in bundle representative of hot bundle steady state power (peak linear power of 46.8 kW/m).*
- *Coolant pressure of 25 MPa.*
- *Coolant inlet temperature of 553 K.*
- *Steady state mass flow rate of 2700 kg/m<sup>2</sup>.s.*
- *Coolant flow decreased to 50% of steady state coolant flow while power remains constant.*

# Properties of supercritical water vary sharply around pseudo-critical temperature ( $p=25$ MPa)



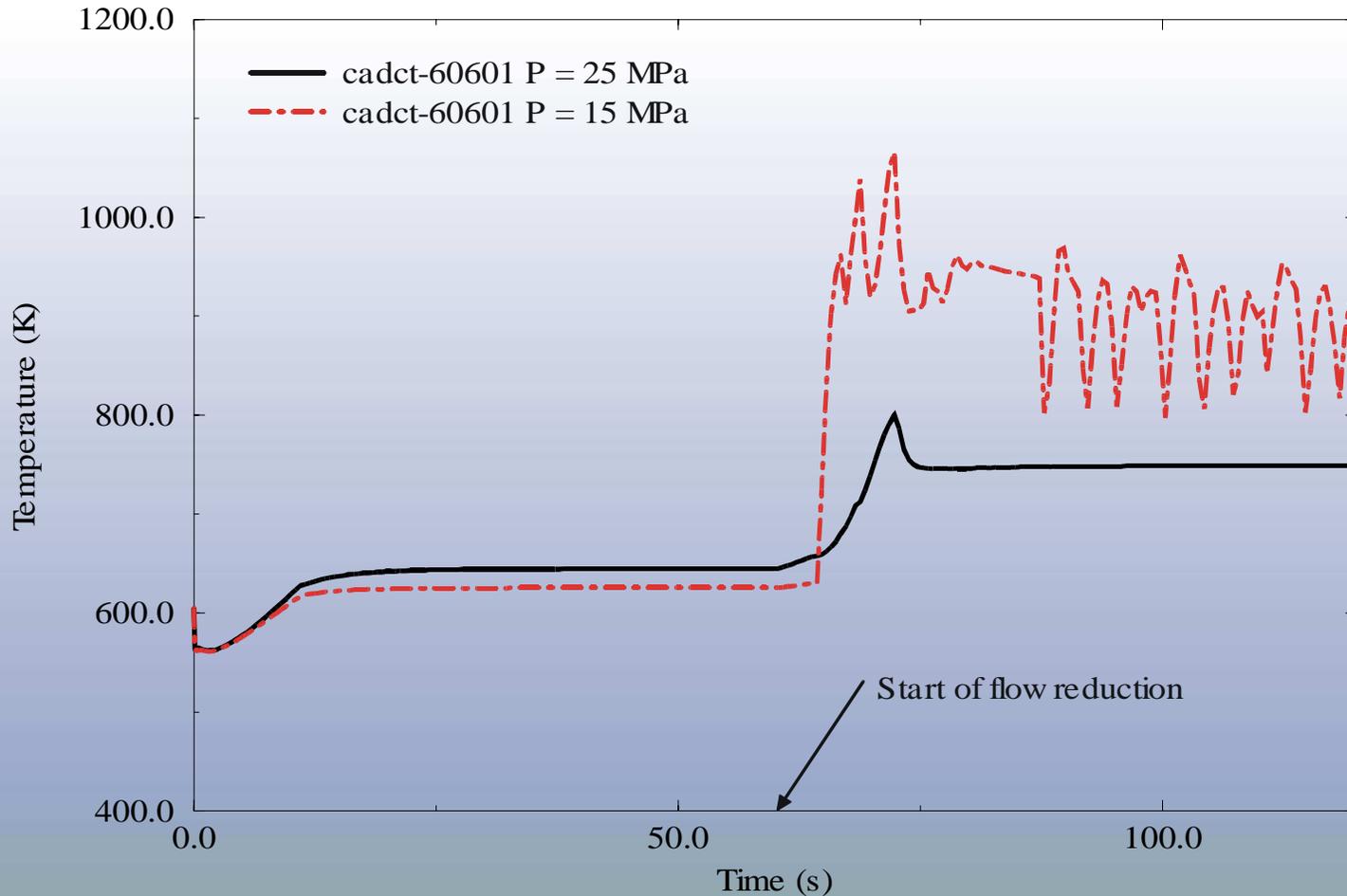
# Variation in thermal conductivity with respect to temperature ( $p=25$ MPa)



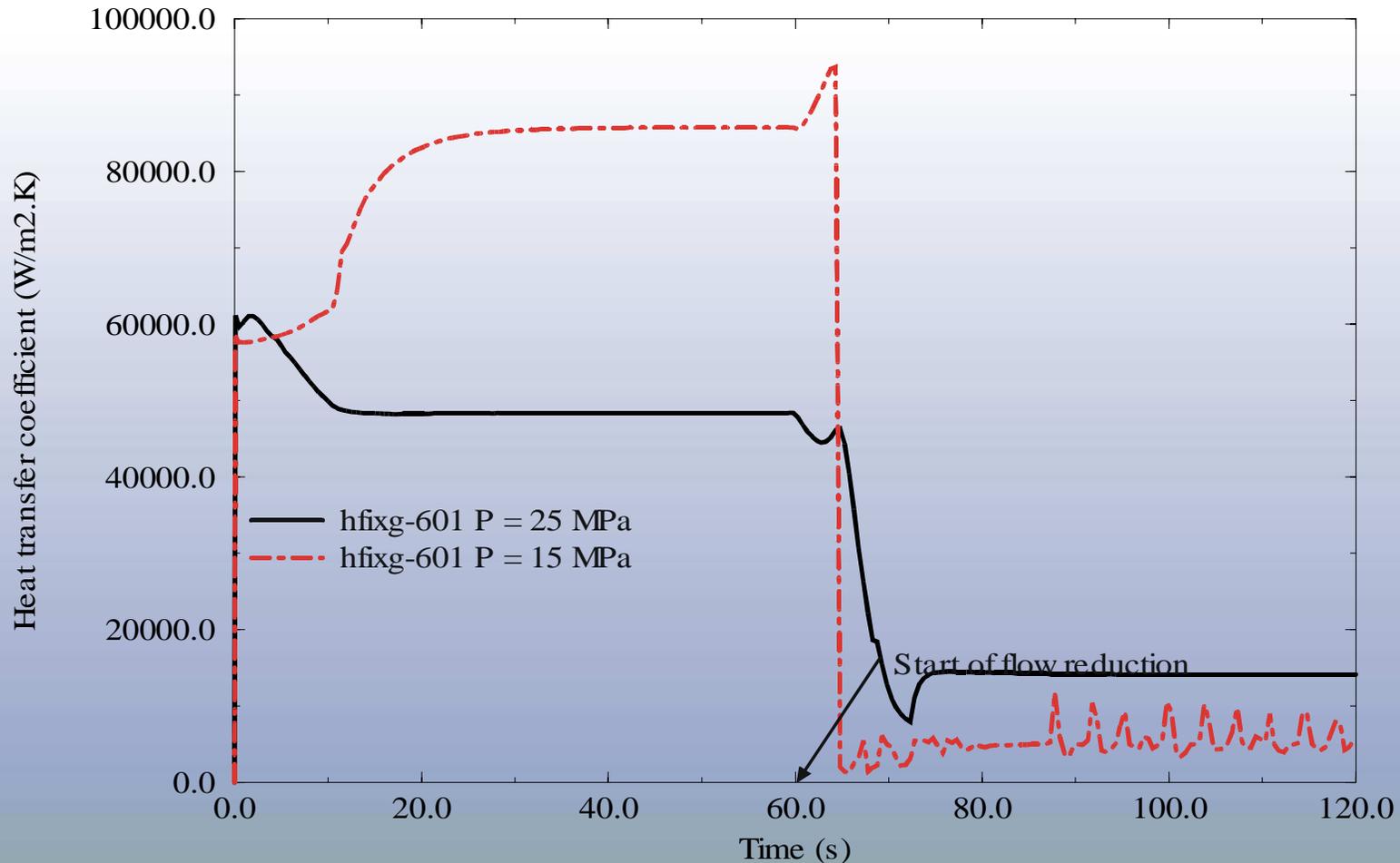
## **Convective heat transfer correlations for supercritical water are summarized by Cheng et al.**

- *Reference: “Thermal-Hydraulic Analysis of Supercritical Pressure Light Water Reactors,” International Congress on Advanced Nuclear Power Plants (ICAPP), Hollywood, Florida, June 9-12, 2002.*
- *Bishop correlation : A A. Bishop, R. O. Sandberg, and L. S. Tong, WCAP-2056-P, Part III-B, February 1964.*
- *Koshizuka correlation based on numerical analysis: “Numerical Analysis of Deterioration Phenomena in Heat Transfer to Supercritical Water”, Int. J. Heat Mass Transfer 38, 3077-3084 (1995).*
- *Dittus-Boelter:  $Nu=0.023Re^{0.8}Pr^{0.333}$*
- *Further experimental work required to identify and improve best correlation and reduce uncertainty in correlation.*

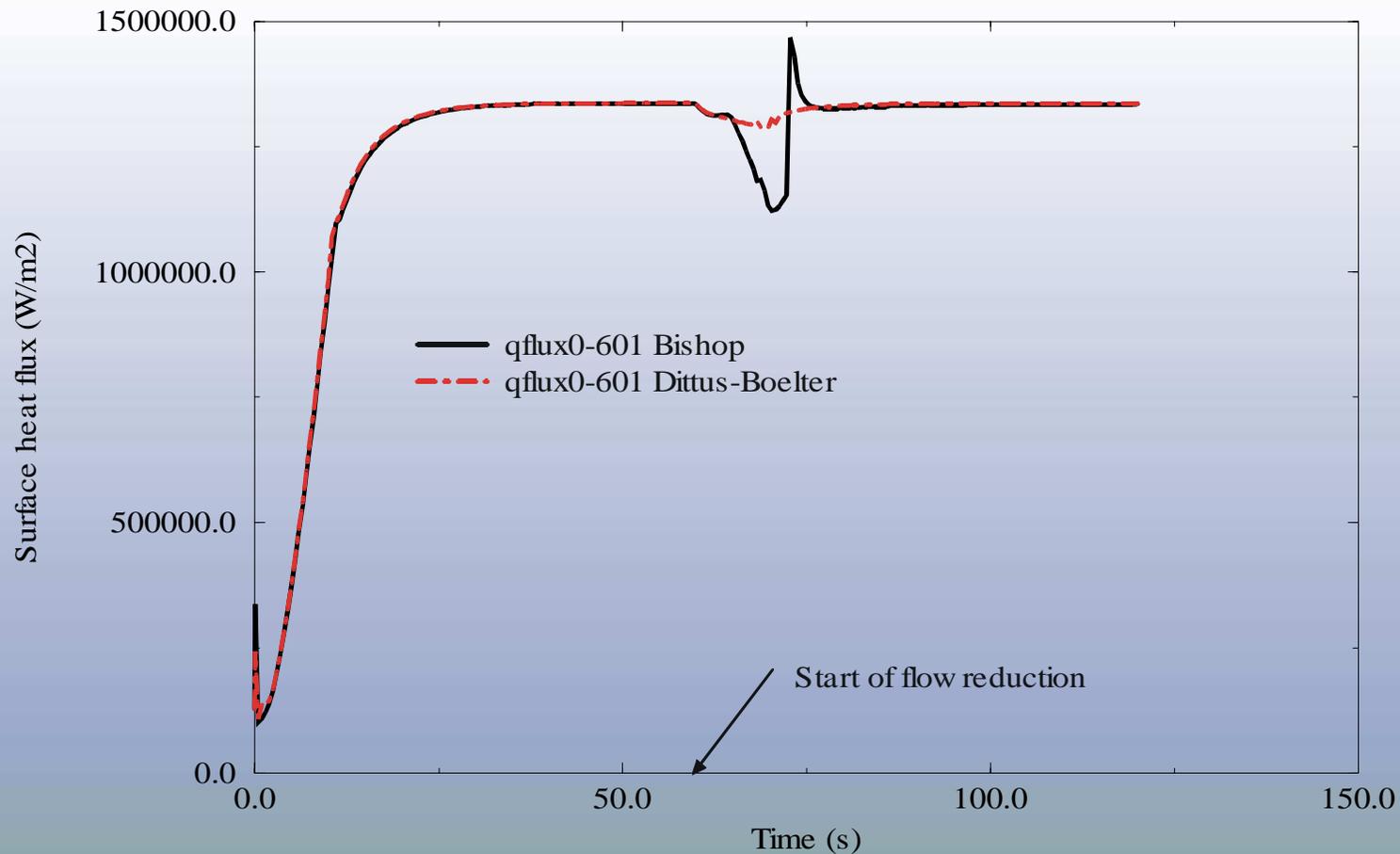
## Greater cladding temperature increase after flow reduction for coolant pressure of 15 MPa than 25 MPa



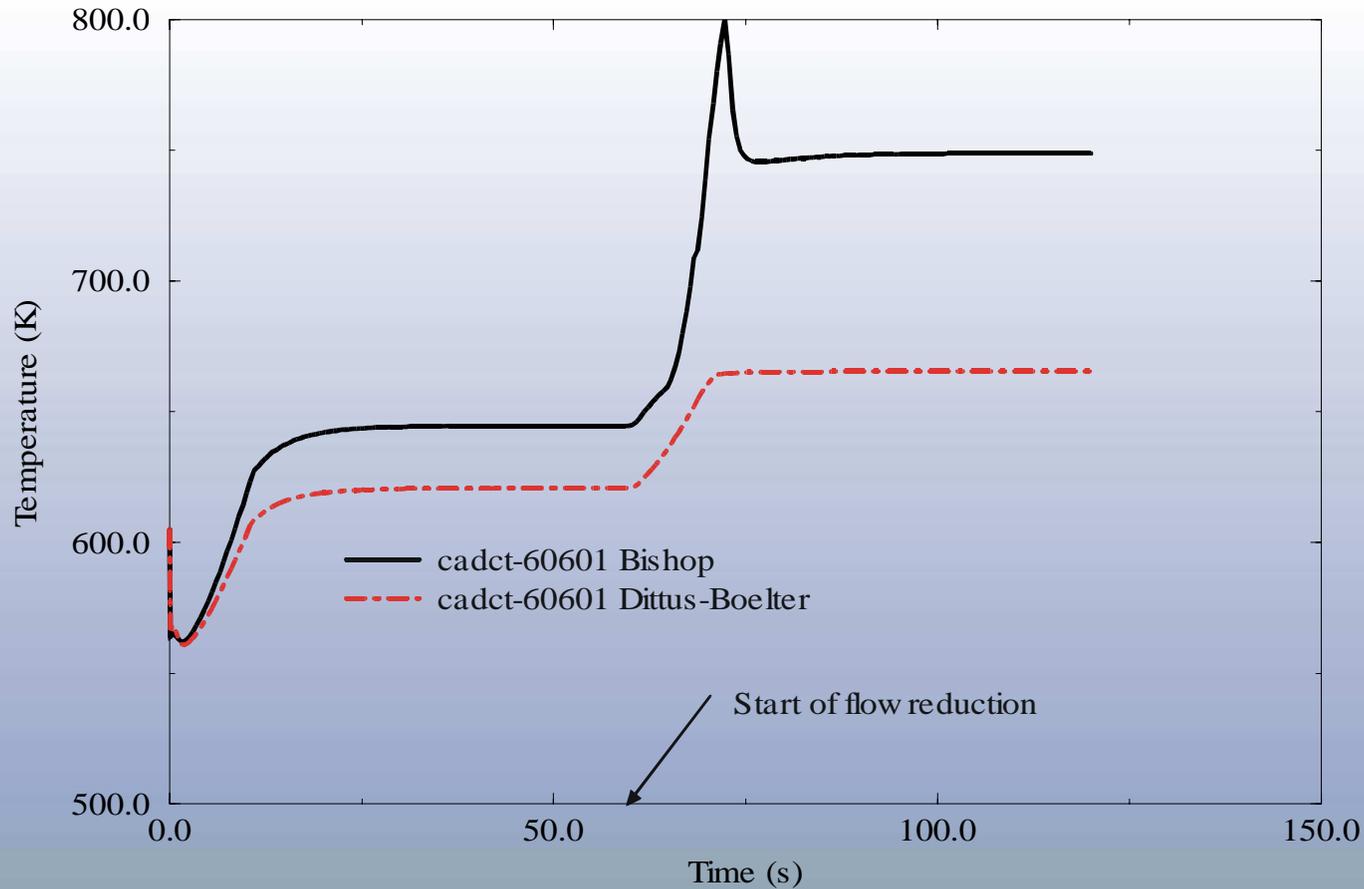
## After flow reduction, convective heat transfer coefficient at 25 MPa coolant pressure is double that at 15 MPa



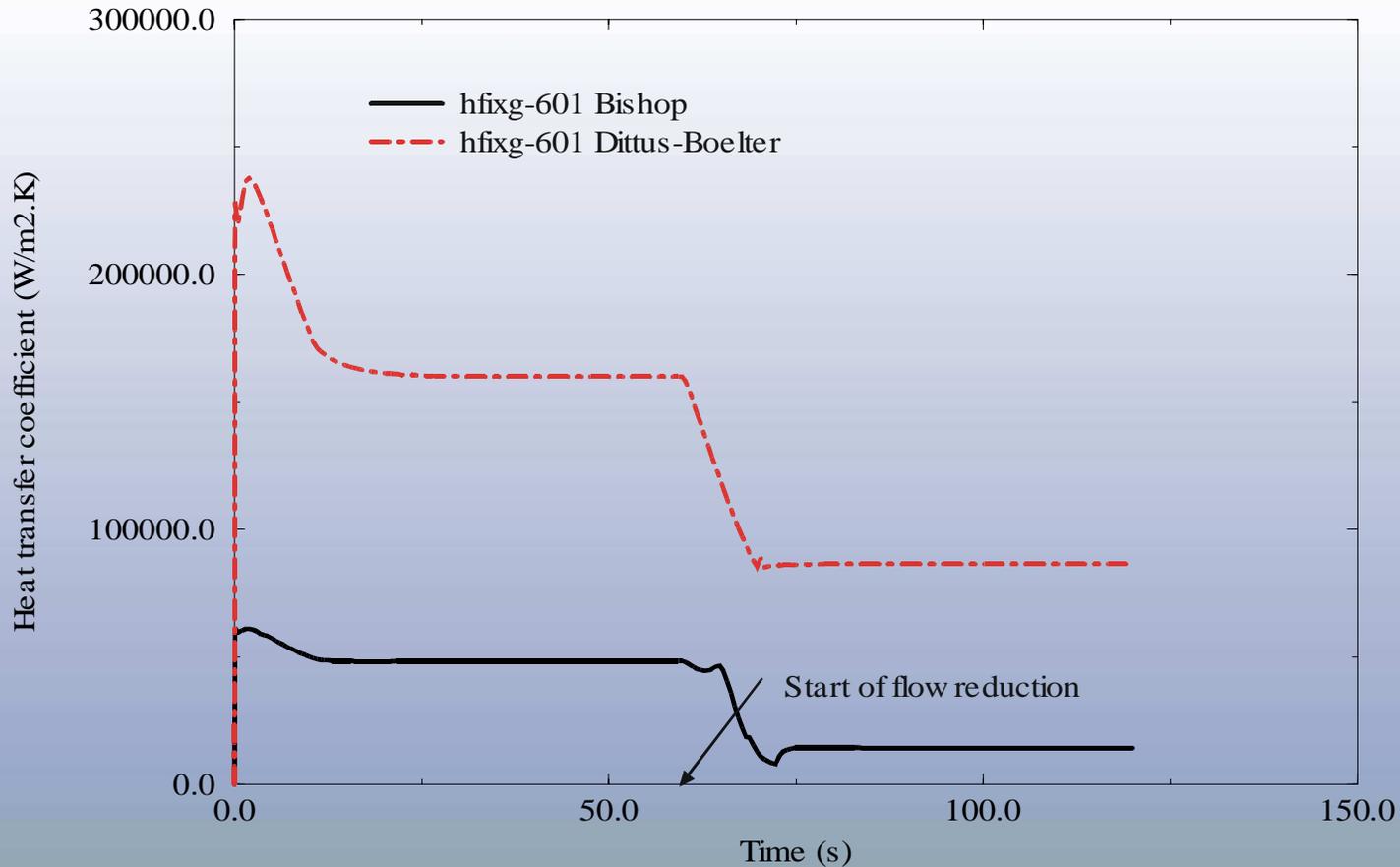
## ***Bishop correlation predicts greater heat flux deterioration following flow reduction than Dittus-Boelter correlation***



## Higher cladding temperature calculated using Bishop correlation than Dittus-Boelter correlation



## ***Bishop correlation calculates more than factor of two smaller heat transfer coefficient than Dittus-Boelter***



# Conclusions

- *SCDAP/RELAP5-3D has been extended to analyze behavior of fuel rods in SCWRs.*
- *Ballooning in SCWR fuel rods delayed and less extensive due to greater strength and less ductility of MA956 cladding at high temperatures.*
- *Temperature increase in SCWR fuel rods during flow reduction less severe than in LWR.*
- *Significant uncertainty in models for convective heat transfer in supercritical water.*